
Effect of Thermal Ageing and Microstructure on
Fracture Mechanical Behavior of Ni-based Alloy
Dissimilar Metal Welds

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Abstract

DMWs are commonly used to join austenitic and ferritic components in the RCPB and can become potential concerns regarding the structural integrity of the nuclear power SSC. In particular, the knowledge on the local strength mismatch at the LAS/nickel-based alloy weld metal interface upon PWHT and during long-term ageing is lacking. The amount of information available in the open literature on the fracture mechanical and microstructural changes occurring at the fusion boundary after buttering, welding, PWHT and long-term ageing is relative limited and often very case dependent. Understanding the fracture mechanical and microstructural appearance, behavior and development of the complex DMW fusion boundary is crucial for both the improvement of nuclear component integrity and to ensure safe long-term operation. In this study, a TVO NG DMW mock-up are studied, thus the test materials are representative of actual power plant components. The project promotes the knowledge transfer, improve the nuclear materials and fracture mechanics competence and strengthen the connections and experience exchange between the Nordic research organizations, universities, industries, authorities, and especially the young generation. The project deals with the structural integrity, long-term operation, and ageing management, which are relevant for both present and future nuclear power plants. The technical results provide a basis for assessment of long-term operation for the Finnish and Swedish nuclear power plants for both the operators and the regulatory perspectives. Dissemination through the peer reviewed publications and the oral presentations at the international conferences ensured the knowledge exchange in international and Nordic networks.

Key words

Dissimilar metal weld, Alloy 52, fusion boundary, characterization, fracture mechanical test

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**Final Report from the NKS-R FEMMA activity
(Contract: AFT/NKS-R(24)134/2)**

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1. Introduction

Dissimilar metal welds (DMWs) are commonly used to join austenitic and ferritic components in the reactor coolant pressure boundary, often using a nickel-based filler metal. DMWs can become potential concerns regarding the structural integrity of the nuclear power systems, structures and components. In particular, the knowledge on the local strength mismatch at the low alloy steel (LAS)/nickel-based alloy weld metal interface upon post-weld heat treatment (PWHT) and during long-term ageing is lacking. The amount of information available in the literature on the fracture mechanical and microstructural changes occurring at the fusion boundary (FB) after buttering, welding, PWHT and long-term ageing is relatively limited and often very case dependent. As a part of the NKS-R program in 2024, VTT, Chalmers University of Technology, and KTH have continued to study the microstructural properties and fracture mechanical performance of an Alloy 52 narrow-gap (NG) DMW, provided by TVO.

The project deals with the structural integrity, long-term operation, and ageing management, which are relevant for both present and future nuclear power plants. The technical results provide a basis for the assessment of long-term operation for the Finnish and Swedish nuclear power plants for both the operators and the regulatory perspectives. In 2024, the low toughness specimens from 15 000 h thermally aged TVO mock-up have been investigated. These specimens mostly have a crack propagation at 0.3 mm from the FB in the LAS heat affected zone (HAZ). Some detailed fractography and cross-sectional microstructure analysis were performed. High-resolution microstructural characterization by means of scanning electron microscopy (SEM) electron backscatter diffraction (EBSD) have been performed at the heat-affected zone (especially from 0.5 mm towards the fusion boundary). Atom probe tomography (APT) measurements on the as-received (AR) and lab thermally aged (TA) (at 400 °C for 15 000 h) conditions of TVO mock-up indicates potential phosphorous segregation during thermal aging.

There are active involvements of young scientists in NKS-FEMMA. The work is mainly executed by three PhD students Noora Hytönen, Laura Sirkiä and Timo Veijola. Young scientists or postdocs, Sebastian Linqvist, David Mayweg, Armin Halilovic and Pedro Ferreiros are the main working group. A new Master Thesis student is under selection at KTH and the thesis work will start in winter 2024. Additionally, young engineers and researchers at TVO, KTH, Forsmark, STUK, OKG, Ringhals, Chalmers and VTT are following the progress of the work in the project meetings and seminar/workshop organized by NKS-FEMMA. This enables knowledge retention and transfer to the young generation of researchers at all participating organizations. Developing and maintaining the knowledge of DMW-related issues is vital for the Nordic nuclear power industry. NKS-FEMMA creates a unique opportunity to keep close contacts between researchers and nuclear industry.

NKS-FEMMA project achieves high standard of academic excellence. A journal articles from NKS-FEMMA project was published in *Materials Characterization* at the end of 2024 (10.1016/j.matchar.2024.114419). Additionally, an oral presentation on the NKS FEMMA 2024 project was delivered in the Nuclear Materials (NuMat) conference held in Singapore during 14–17 October 2024.

2. Mechanical investigations on DMWs

The macrostructure of the DMW mock-up studied here, is presented in Figure 1(a). The HAZ adjacent to the FB is estimated to be about 2.5 mm wide. Figure 1(b) illustrates the variation in micro-hardness and prior austenite grain (PAG) size as a function of distance from the FB in the LAS HAZ under both AR and TA conditions. The hardness reaches a maximum and the grain size a minimum at 0.3–0.5 mm from the FB, *i.e.* in the grain-refined region. For the T_0 fracture toughness TA specimens, with crack propagation occurring in the HAZ (~0.3–0.5 mm from the FB) without deviation to the FB, a significantly lower (~1/3) fracture toughness was observed in comparison to specimens where the notches were placed near the FB and with the cracking path deviating to the FB. A summary of the T_0 testing results and representative cross-sectional characterizations is shown in Figure 1(c-e). While the FB in the DMW is typically considered the most vulnerable location regarding fracture mechanical behavior, this study revealed a much lower local brittle fracture toughness in the HAZ at a distance of approximately 0.3–0.5 mm from the FB.

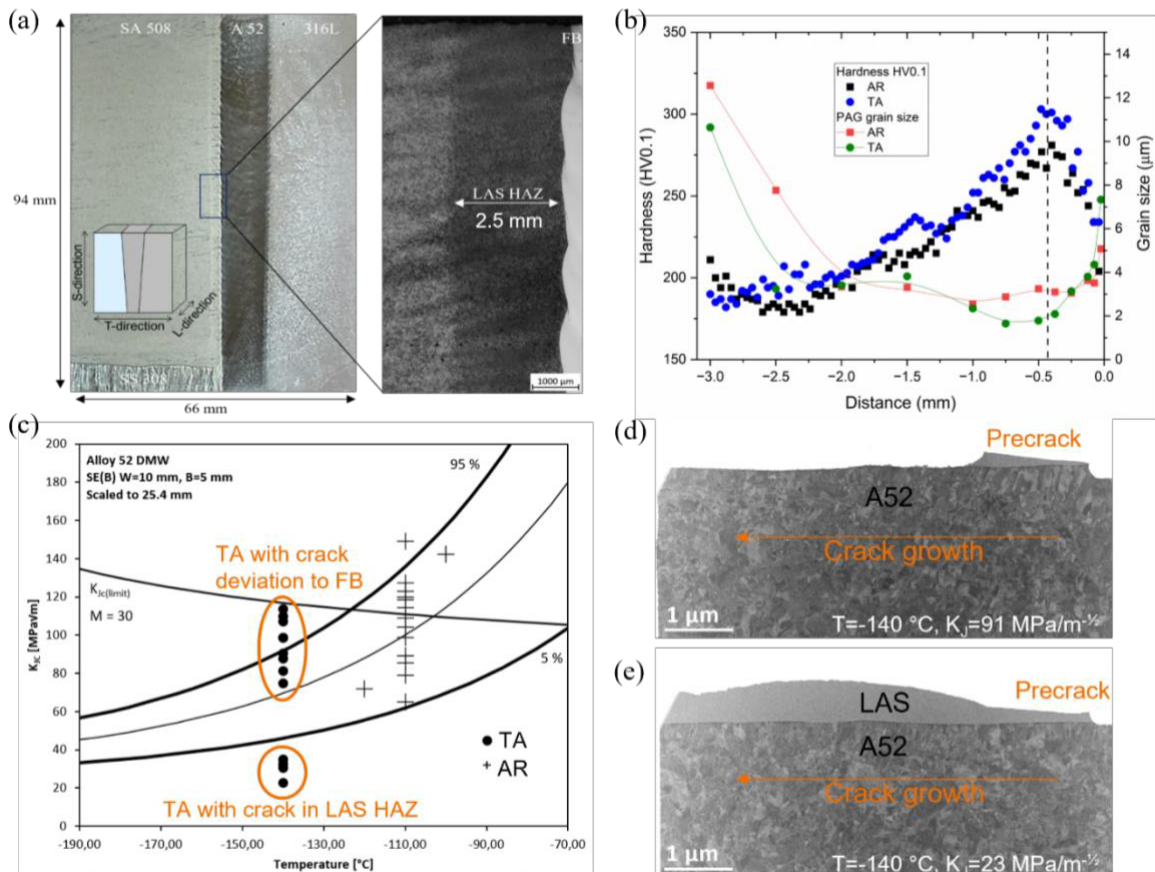


Figure 1: (a) The investigated NG gas tungsten arc welding (GTAW) DMW mock-up. (b) The change in micro-hardness and PAG grain size in the HAZ of LAS adjacent to the FB. (c) Summary of the T_0 testing results for AR and TA materials. The representative cross-sectional characterizations of TA T_0 testing specimens (d) with and (e) without crack deviation to the FB. The deviation to the FB resulted in a much higher fracture toughness.

Representative fracture surfaces of the T_0 fracture toughness specimens in the AR and TA materials are shown in Figure 2. The AR material exhibits brittle fracture with cleavage facets, while the TA material shows intergranular cracking. This reveals that long-term thermal aging increases grain boundary (GB) decohesion and intergranular cracking, resulting in a decreased fracture toughness in the LAS HAZ of the TA specimens.

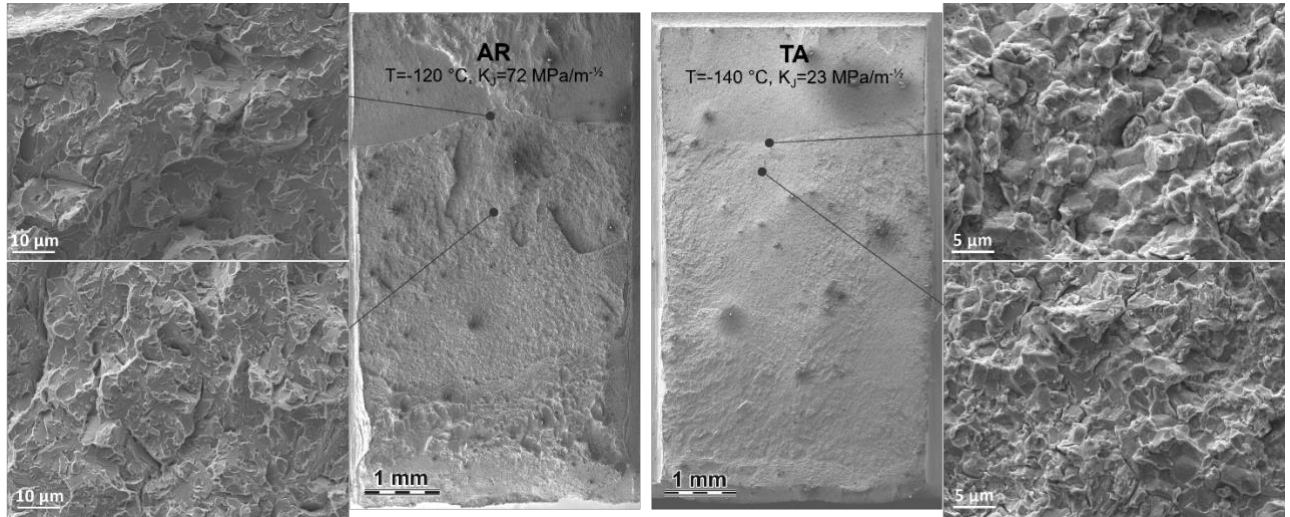


Figure 2: Representative fracture surfaces of the T_0 fracture toughness specimens in the AR and TA materials.

The carbide distribution in the HAZ (at a distance of approximately 0.3 mm from the FB) of the AR and TA materials is illustrated in Figure 3. In the HAZ, the volume fraction of carbide increases upon thermal aging. Analytical scanning transmission electron microscopy (STEM), energy-dispersive X-ray spectroscopy (EDS) maps and extracted elemental line profiles (Figure 3(a)) reveal two types of carbides in the LAS HAZ of the AR material. The dominant and larger carbides are $(\text{Fe, Mn, Cr})_3\text{C}$ carbides (θ - M_3C type), located both at GBs and within grains, with lengths ranging from tens to 150 nm and widths of 30–80 nm. Smaller carbides are Mo-rich (Mo_2C), located at GBs with a few tens of nm in size. In the HAZ of the TA material, the large carbides remain $(\text{Fe, Mn, Cr})_3\text{C}$, with lengths extending up to 350 nm, as shown in the STEM-EDS maps in Figure 3(b). Most Mo-rich carbides are still with sizes around a few tens of nm, though some exceed 100 nm. Thermal aging did not alter the types of carbides but resulted in their coarsening in the HAZ.

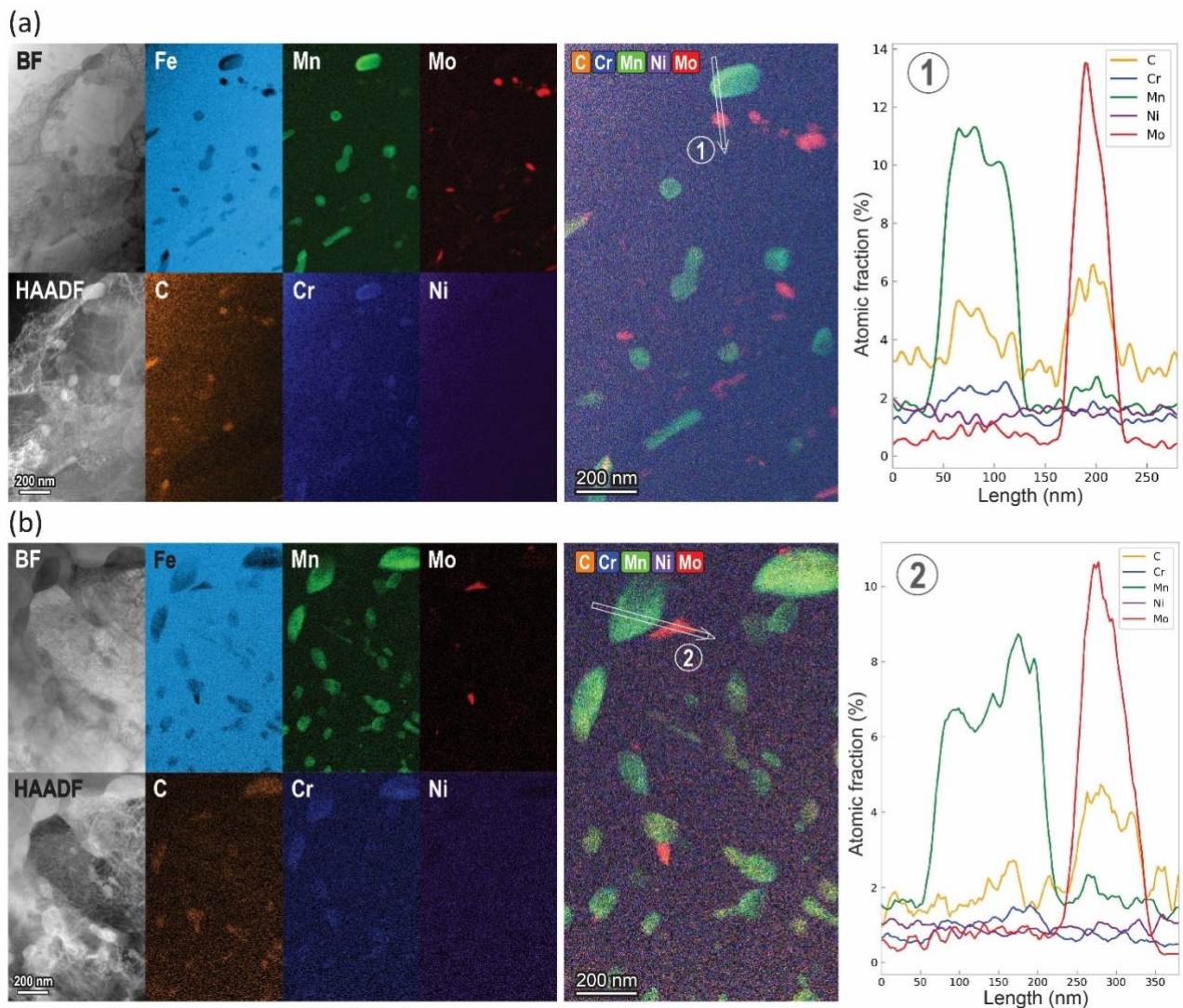


Figure 3: Carbides in the LAS HAZ of (a) AR and (b) TA materials. The illustrative images are bright field (BF) images, high-angle annular dark-field (HAADF) images, STEM-EDS elemental maps and extracted line profiles.

3. Microstructural assessment using APT

Multiple APT measurements were conducted on both AR and the TA materials. Representative reconstructions of APT measurements from AR and TA materials are shown in Figure 4(a-b). In the AR condition, there is only a very limited tendency for P to segregate at GBs, whereas significant P segregation is observed in the TA material. Other elements, such as Mo, C, and Mn, are found to segregate at GBs in both AR and TA materials. Figure 5(a-b) display 1D concentration profiles across two GBs, showing the absence of P in the AR GB but up to 0.3 at.% P in the TA GB. Mo, Mn, and C are segregated in both AR and TA materials, but their concentrations at TA GBs are markedly higher.

Moreover, more and larger carbides can be found at GBs in the TA material in comparison to the AR material, as APT analysis of AR specimens showed no visible carbides associated with GBs, whereas significant carbides were observed in needles from TA specimens. Similar observations were made by the STEM analysis. A proximity histogram displaying the concentration of various elements at progressive distances from the surface of the Mo-carbide

is presented in Figure 5(c). This plot indicates the composition inside the carbide, with approximately 60 at.% Mo and 30 at.% C, suggesting it is a Mo_2C -type carbide. Notably, Mo and C were found clustering along elongated features (likely dislocations or disconnections) in the AR material, as visible in the right box in Figure 4(a). This might highlight the tendency of these two elements to react and form carbides, potentially representing a nucleation stage for the Mo_2C carbides found in the TA material. Additionally, Cu-rich clusters, upon long-term aging, were not detected in this study. This discrepancy may be due to the lower Cu content in the current study (0.06 vs. 0.098 wt%).

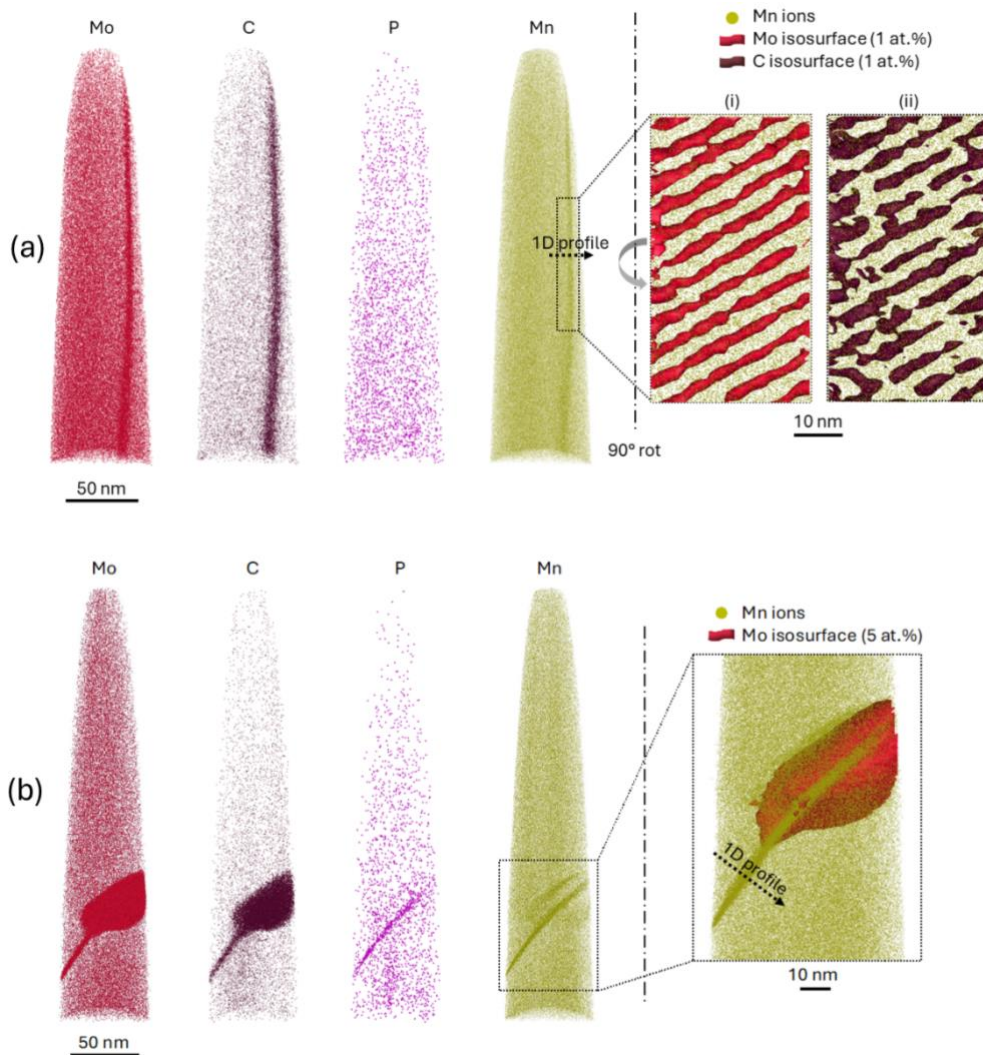


Figure 4: 3D reconstruction of APT data collected across a high-angle GB (HAGB) in (a) AR material and (b) TA material. Atom maps of Mo, C, P and Mn are plotted. In (a), segregation of Mo, C and Mn (but no P) is evident from these maps. Segregation of Mo and C follows a pattern fitting enrichment at dislocations or disconnections arranged in a tight array as visible in the boxes to the right (Mo isosurfaces shown in (i), C isosurfaces shown in (ii)). In (b), segregation of Mo, C, P and Mn is evident from these maps. The presence of a Mo-carbide at the GB is evident, a detail of this feature is presented in the box to the right. The location and direction of 1D profiles (see Figure 5) are indicated by the dashed arrows.

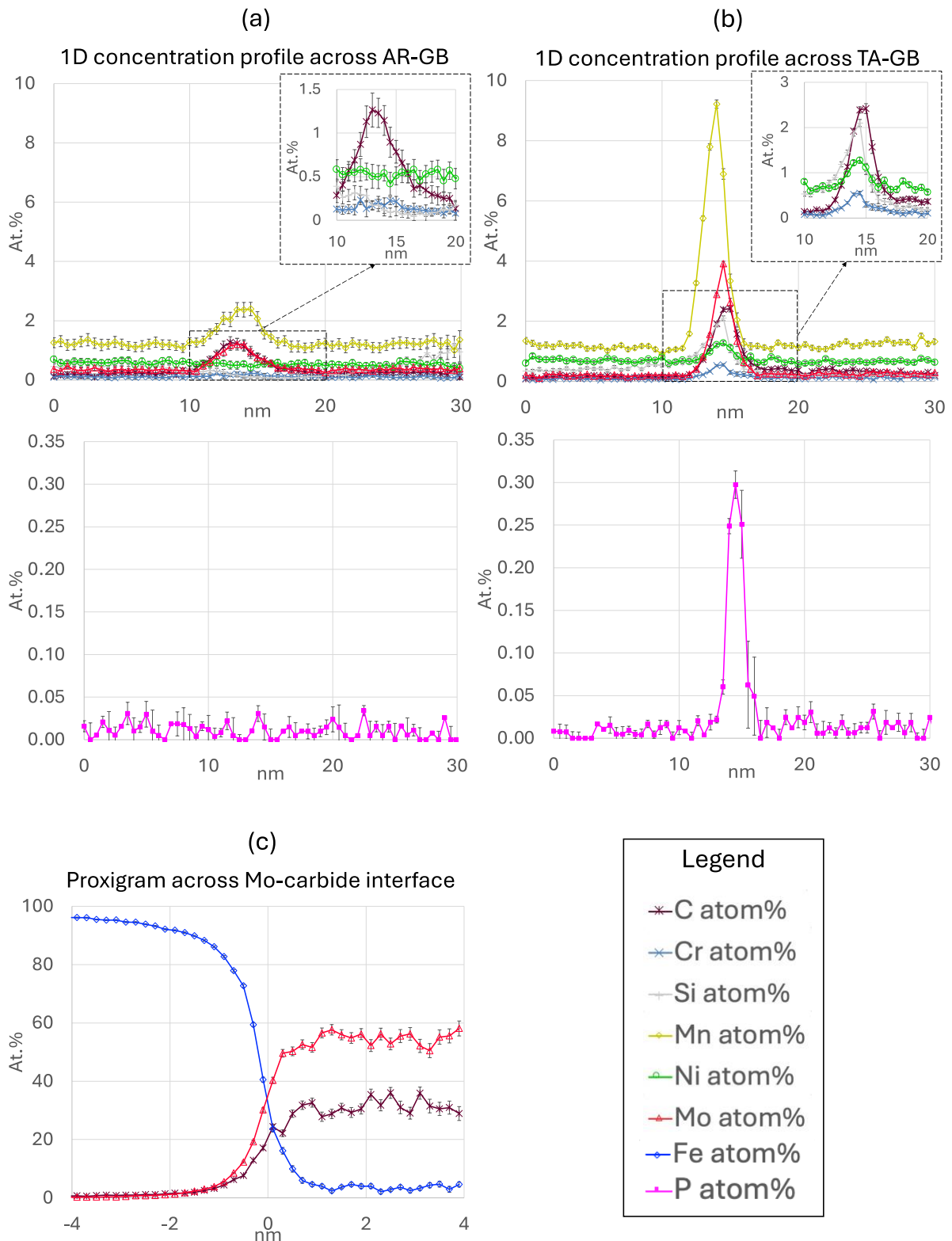


Figure 5: 1D composition profile across HAGB in the AR material (a), main alloying elements are presented in the top plot while P is presented in the bottom plot. 1D composition profile across HAGB in the TA material (b), main alloying elements are presented in the top plot while P is presented in the bottom plot. Proximity histogram measured across the matrix-carbide interface for the carbide found in the TA material (c).

4. Discussions

Owing to the repulsive interactions and segregation binding, the enrichment of P upon aging time results in a depletion of prior-segregated cohesion-enhancing solute elements at GBs, which critically influence the P-induced temper embrittlement. However, from the current experimental results, it appears that the segregation of Mn, Mo and C in the AR condition does not effectively repel P atoms from GB segregation upon extended tempering. On the other hand, when the segregation energy of one element is weak, such as Al in a ferritic steel, the strong repulsive interactions of C-Al and B-Al were sufficient to repel Al from GBs. Strong C segregation is usually considered to suppress P-induced embrittlement, based on the C and P site competition mechanism, or the C-enhanced GB cohesion mechanism. Moreover, Mo is considered an element enhancing GB cohesion. In the present work, however, when C segregates to GBs and facilitates the formation and coarsening of carbides, it seems to weaken the resistance to temper embrittlement.

The reduction in brittle fracture resistance of DMWs can arise from inherent local inhomogeneities and increased constraint. This indicates that in the TA specimens, where the crack propagation occurred in the LAS HAZ at a distance of ~0.3–0.5 mm from the FB, the elemental segregation and carbide coarsening upon thermal aging contributed to the low resistance to brittle fracture. Long-term thermal aging at 400 °C for 15 000 h leads to coarsening of carbide precipitates and significant P and Mn segregation to GBs in the HAZ (despite a low bulk P content in modern low-P LASs). The carbide coarsening and segregation result in the presence of local brittle zones (additionally associated with peak nano-hardness and minimal grain size in HAZ), a higher degree of intergranular cracking and a low fracture toughness in the HAZ at a distance of ~0.3–0.5 mm from the FB. The direct observations of significant elemental segregation to GBs and the resultant reduction in fracture toughness in the HAZ are unexpected for modern low-P LASs. This should be considered for evaluating the long-term structural integrity of nuclear components.

5. Conclusions

Long-term thermal aging results in significant P and Mn segregation to GBs (despite a very low bulk P content) and coarsens the carbides in the HAZ, which enhances GB decohesion, leads to local brittle zones, facilitates brittle crack propagation along GBs and ultimately causes a low fracture toughness.

P can segregate to GBs in LAS after long service times even when its bulk content is very low. Simultaneously, Mn segregates at GBs and contributes to the intergranular fracture. It is observed that when C segregates to GBs and facilitates the formation and coarsening of GB carbides, it weakens the resistance to temper embrittlement.

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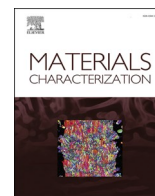
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Unexpected thermal aging effect on brittle fracture and elemental segregation in modern dissimilar metal weld

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ABSTRACT

A full-scale dissimilar metal weld safe-end mock-up, precisely replicating a critical component of a modern nuclear power plant, was investigated. The brittle fracture behavior, carbide evolution and nanoscale elemental segregation in the heat-affected zone (HAZ) of low alloy steel (LAS) were analyzed under both post-weld heat-treated and thermally-aged conditions (400 °C for 15,000 h, equivalent to 90 years of operation) using analytical electron microscopy and atom probe tomography. The observed increase in grain boundary (GB) decohesion and intergranular cracking on the fracture surface and the decrease of fracture toughness are primarily attributed to P and Mn segregation to GBs and the coarsening of carbides upon long-term thermal aging. The direct observations of significant elemental segregation to GBs and the consequent reduction in fracture toughness in the HAZ are unexpected for modern low-phosphorus LASs, highlighting potential concerns for evaluating the structural integrity of modern nuclear power plants.

1. Introduction

Dissimilar metal welds (DMWs) are routinely employed to join bainitic/ferritic low alloy steel (LAS) reactor pressure vessel with austenitic stainless steel pipes using Ni-based filler metals [1–3]. The differing chemical composition, crystal structure and mechanical properties at the fusion boundary (FB) create a mismatch [4–7]. DMWs can become a concern for the structural integrity in nuclear power systems, structures and components (SSC). A couple of failures have been observed in DMWs in nuclear power plants [8–10]. The knowledge on the fracture behavior, cracking behavior and microstructural changes occurring at the DMW interface upon long-term plant operation and aging is lacking.

Carbides formed in the heat-affected zone (HAZ) of LAS can serve as fracture initiators, significantly impacting brittle fracture behavior [11,12]. Furthermore, phosphorus segregation to grain boundaries (GBs) and precipitates (e.g. carbides) can increase cracking susceptibility, promote GB decohesion and intergranular fracture, and elevate the ductile-to-brittle transition temperature (DBTT) [13–15]. Consequently, the P content in modern reactor LASs has been limited to 0.008 wt%. Recently, Lindqvist et al. [16,17] reported that in a modern SA508/Alloy 52 DMW, crack initiation and propagation in thermally-

aged T₀ fracture toughness specimens occurred in the HAZ without the crack deviating to the FB, resulting in a significantly lower fracture toughness than for the specimens where the notch crack was placed near the FB and the crack path deviated to the FB. Understanding the elemental segregation and carbide evolution occurring in the HAZ of modern reactor LASs upon long-term thermal aging is critical for evaluating structural integrity and ensuring the long-term operation of nuclear components [18–21]. However, such information is scarce in the open literature [22–24].

In this paper, a full-scale DMW safe-end mock-up, precisely replicating a critical component of a modern nuclear power plant, was studied. The brittle fracture behavior, carbide evolution and nanoscale elemental segregation in the LAS HAZ were analyzed under post-weld heat-treated and thermally-aged conditions (equivalent to 90 years of operation) using analytical electron microscopy and atom probe tomography (APT). The intrinsic interaction of nanoscale elemental segregation and carbide coarsening with the GB decohesion and the decrease of fracture toughness was discussed.

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2. Materials and method

In the present study, a 1:1 scale narrow-gap gas tungsten arc welding (GTAW) DMW mock-up, joining an SA508 Gr.2 nozzle with a 316LN safe-end using Alloy 52 filler metal, representative of a critical component in a modern Nordic nuclear power plant [25–27], is studied. The chemical composition of the materials is provided in Table 1. The mock-up is welded one bead per layer by gas tungsten arc welding (GTAW). The mock-up was post-weld heat treated at 550 °C for 15 h and 610 °C for 7 h. The studied conditions include the as-received post-weld heat-treated condition (“AR”) and the long-term thermally-aged condition of 15,000 h at 400 °C (“TA”), purposely equivalent to 90 years of operation at 288 °C in a nuclear power plant [15].

For characterization of the fracture toughness T_0 reference temperature for the DBTT region according to ASTM E1921, $5 \times 10 \times 50$ mm single edge-notched side-grooved bend (SE(B)) specimens were tested. The cracks of the SE(B) specimens are nominally located in the HAZ of the LAS, 0–350 μ m from the FB. The cracks are parallel to the FB. Before testing, the specimens were pre-cracked by fatigue to the initial crack length over a specimen width ratio a_0/W of 0.5, using a RUMUL resonant testing machine. The maximum value of the applied stress intensity factor, K_{max} , was kept below 15 MPa \sqrt{m} during the fatigue pre-cracking. The fracture toughness testing was performed using an MTS universal servo-hydraulic testing machine equipped with a 10 kN load cell. The crack mouth opening displacement (CMOD) was measured using an Epsilon 3541-003 M-040 M-LHT clip gauge. After testing, the specimens were soaked in liquid nitrogen and broken into two halves to measure the crack lengths. The crack length and crack front straightness were checked before calculation of T_0 . The quality of the results was assessed after testing. All the T_0 specimens fractured by the brittle fracture mechanism and the brittle fracture initiated without significant ductile crack growth.

The prior austenite grain (PAG) size was evaluated with electron backscatter diffraction (EBSD). A Zeiss Crossbeam 540 scanning electron microscope (SEM) with an EDAX HikariPlus EBSD detector was applied. EBSD was performed with a working distance of 14–15 mm, a step size of 150 nm and with the samples tilted at 70°. The resulting EBSD inverse pole figure (IPF) was analyzed with the TSL OIM Analysis 8.6 software. The Vickers micro-hardness measurements with loads of 0.1 kg (HV0.1) across the FB were performed using a Struers DuraScan-80 device.

To assess the microstructure and the carbides in the LAS HAZ, lamellae were prepared using focused ion beam (FIB) with JEOL JIB 4700. The lamellae were investigated using a Thermo Fisher Talos F200X analytical transmission electron microscope (TEM), equipped with the Super-X Energy-dispersive X-ray spectroscopy (EDS) system, operating at 200 kV. High-angle annular dark-field (HAADF) and bright field (BF) images were recorded.

For nanoscale chemical analysis of GBs, APT was performed. EBSD maps collected on a TESCAN GAIA3 equipped with an Oxford-NordlysNano detector were used to investigate the GB misorientations on AR and TA samples. This allowed to identify sites of interest for the APT lift-outs – high-angle GBs with comparable misorientation angles (55–60°). Site specific APT sample preparation was conducted on a dual-beam FIB-SEM FEI Versa 3D workstation. The back-scattered electron signal was used to find the previously identified sites of interests, and to guide during the tip sharpening step allowing to place the targeted GB in the proximity of the needle apex. Transmission Kikuchi Diffraction (TKD) was performed on the sharpened specimens to confirm the presence of the GBs and to verify their misorientation angles before APT

measurement. TKD was performed on a TESCAN GAIA3, in STEM mode, equipped with Oxford-NordlysNano detector. All APT measurements were conducted on a CAMECA LEAP 6000 XR in laser mode, at 50 K specimen temperature, 50–70 pJ laser energy, 0.3 % evaporation rate. Auto pulse frequency control was set to collect a minimum mass-to-charge ratio of 150 Da. The CAMECA APT Suit 6 software was employed to create reconstructions and evaluate the data.

3. Result and discussion

The EBSD maps revealing the grain structure in the HAZ as a function of the distance to the FB in the AR and TA samples are shown in Fig. 1. The macrostructure of the DMW mock-up studied is presented in Fig. 2 (a). The HAZ adjacent to the FB is estimated to be about 2.5 mm wide. Fig. 2(b) illustrates the variation in micro-hardness and PAG size as a function of distance from the FB in the LAS HAZ under both AR and TA conditions. The hardness reaches a maximum and the grain size a minimum at 0.3–0.5 mm from the FB, i.e. in the grain-refined region. For the T_0 fracture toughness TA specimens, with crack propagation occurring in the HAZ (~0.3–0.5 mm from the FB) without deviation to the FB, a significantly lower (~1/3) fracture toughness was observed in comparison to specimens where the notches were placed near the FB and with the cracking path deviating to the FB. A summary of the T_0 testing results and representative cross-sectional characterizations is shown in Fig. 2(c–e). While the FB in the DMW is typically considered the most vulnerable location regarding fracture mechanical behavior, this study revealed a much lower local brittle fracture toughness in the HAZ at a distance of approximately 0.3–0.5 mm from the FB.

Representative fracture surfaces of the T_0 fracture toughness specimens in the AR and TA materials are shown in Fig. 3. The AR material exhibits brittle fracture with cleavage facets, while the TA material shows intergranular cracking. This reveals that long-term thermal aging increases GB decohesion and intergranular cracking, resulting in a decreased fracture toughness in the LAS HAZ of the TA specimens.

The carbide distribution in the HAZ (at a distance of approximately 0.3 mm from the FB) of the AR and TA materials is illustrated in Fig. 4. In the HAZ, the volume fraction of carbide increases upon thermal aging. Analytical scanning transmission electron microscopy (STEM), EDS maps and extracted elemental line profiles (Fig. 4(a)) reveal two types of carbides in the LAS HAZ of the AR material. The dominant and larger carbides are (Fe, Mn, Cr)₃C carbides (θ-M₃C type), located both at GBs and within grains, with lengths ranging from tens to 150 nm and widths of 30–80 nm. Smaller carbides are Mo-rich (Mo₂C), located at GBs with a few tens of nm in size. In the HAZ of the TA material, the large carbides remain (Fe, Mn, Cr)₃C, with lengths extending up to 350 nm, as shown in the STEM-EDS maps in Fig. 4(b). Most Mo-rich carbides are still with sizes around a few tens of nm, though some exceed 100 nm. Thermal aging did not alter the types of carbides but resulted in their coarsening in the HAZ.

The effect of long-term thermal aging on the segregation of P and other alloying elements in the HAZ of LAS was investigated with APT. Given that the extent of segregation is closely linked to the misorientation angle of the GBs [28,29], this study focuses on comparing the elemental concentrations at high-angle GBs with similar misorientation (55–60°) located ~0.3 mm from the FB in both AR and TA conditions. Site specific APT needles were prepared with the FIB lift-out technique [30]. TKD was performed on the sharpened specimens to verify the GB misorientation angles before APT measurements, with the resulting representative maps shown in the Fig. 5.

Table 1
Chemical composition of the Alloy 52 weld and the DMW LAS nozzle (wt%).

	C	Al	Si	P	S	Ti	Cr	Mn	Fe	Ni	Cu	Mo
Alloy 52	0.023	0.66	0.15	<0.005	0.0007	0.54	29.9	0.26	10.4	Bal.	<0.01	<0.01
SA508	0.18	0.02	0.19	0.005	0.001	0.002	0.12	1.49	Bal.	0.78	0.06	0.49

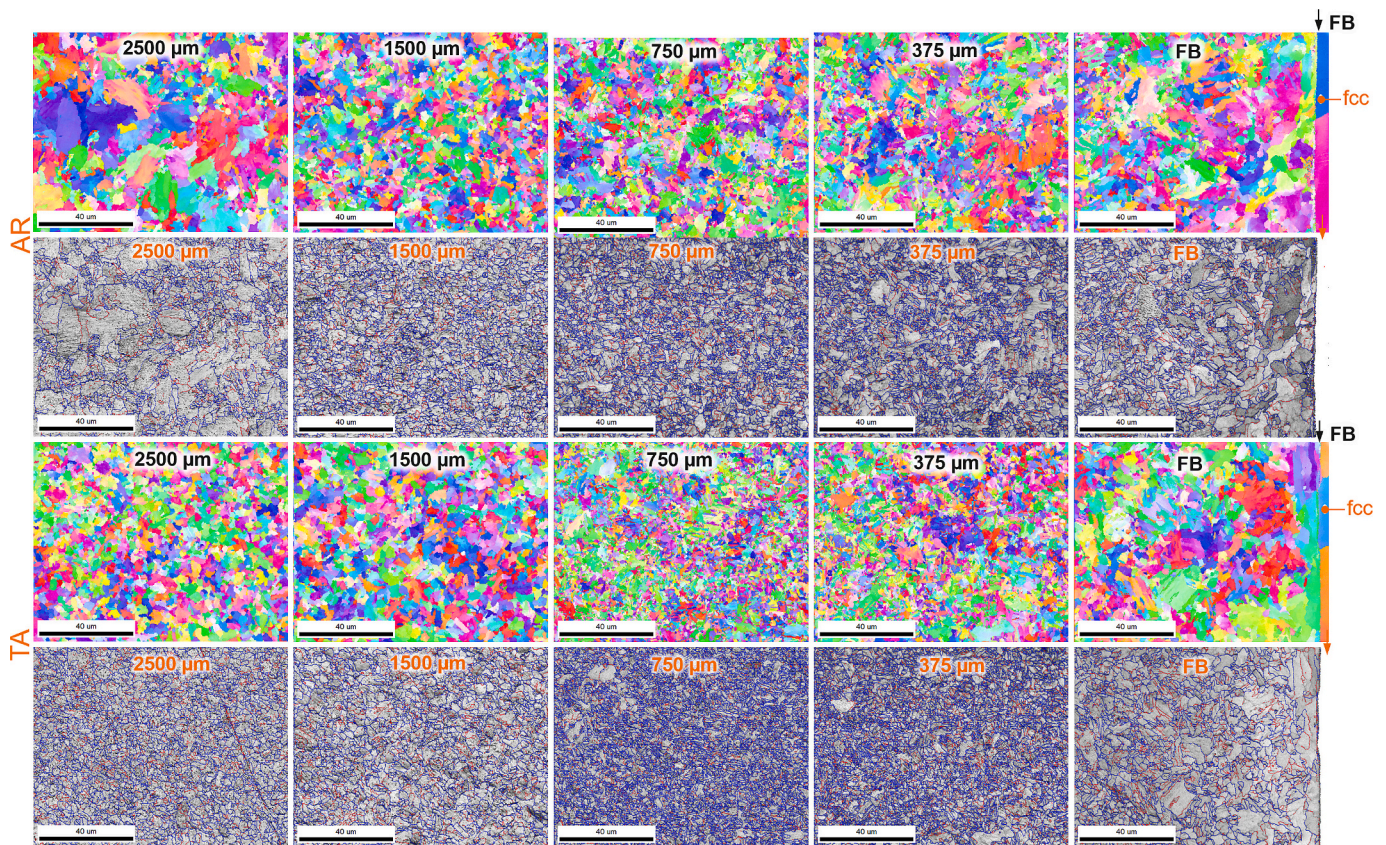


Fig. 1. EBSD IPF and image quality maps (rotation angle $2\text{--}15^\circ$ in red and $>15^\circ$ in blue) per distance to the FB in the AR and TA samples. (For interpretation of the references to colour in this figure legend, the reader is referred to the web version of this article.)

Multiple APT measurements were conducted on both AR and the TA materials. Representative reconstructions of APT measurements from AR and TA materials are shown in Fig. 6(a-b). In the AR condition, there is only a very limited tendency for P to segregate at GBs, whereas significant P segregation is observed in the TA material. Other elements, such as Mo, C, and Mn, are found to segregate at GBs in both AR and TA materials. Fig. 7(a-b) display 1D concentration profiles across two GBs, showing the absence of P in the AR GB but up to 0.3 at.% P in the TA GB. Mo, Mn, and C are segregated in both AR and TA materials, but their concentrations at TA GBs are markedly higher.

Moreover, more and larger carbides can be found at GBs in the TA material in comparison to the AR material, as APT analysis of AR specimens showed no visible carbides associated with GBs, whereas significant carbides were observed in needles from TA specimens. Similar observations were made by the STEM analysis. A proximity histogram displaying the concentration of various elements at progressive distances from the surface of the Mo-carbide is presented in Fig. 7(c). This plot indicates the composition inside the carbide, with approximately 60 at.% Mo and 30 at.% C, suggesting it is a Mo_2C -type carbide. Notably, Mo and C were found clustering along elongated features (likely dislocations or disconnections [31]) in the AR material, as visible in the right box in Fig. 6(a). This might highlight the tendency of these two elements to react and form carbides, potentially representing a nucleation stage for the Mo_2C carbides found in the TA material. Additionally, Cu-rich clusters, as observed by Medouni et al. [32] upon long-term aging, were not detected in this study. This discrepancy may be due to the lower Cu content in the current study (0.06 vs. 0.098 wt%).

Gibbsian interfacial excess (Γ) [33] for P, C, Mo, and Mn segregating at the GB of the AR and TA samples respectively has been calculated and reported in Table 2. As displayed by the 1D profiles and the Gibbsian interfacial excess, the segregation at the measured GBs is significant

already in the AR sample. After TA, all elements accumulate further, particularly Mn and Mo. While C is expected to sit at interstitial sites, P, Mo and Mn are likely to be substitutional [34,35]. Mo plus Mn atoms reach a total value of Gibbsian interfacial excess around 16 atoms/ nm^2 . As comparison, the planar atomic densities of Fe-BCC (100) and Fe-FCC (111) are 12.17 atoms/ nm^2 and 18.74 atoms/ nm^2 , respectively. Overall, after TA it is reasonable to imagine Mn and Mo forming a monoatomic layer at the GB. This can influence the mechanical behavior of the GB and affect the chemical environment potentially favoring the P segregation.

The results are consistent with our previous study [27], showing that the bulk P content in the HAZ grain interior is slightly higher in the AR material compared to the TA material, due to P segregation from the grain interior to GBs after thermal aging. The current APT analysis allows for calculating an approximate concentration of P in the grain interior, excluding contributions from GBs. These results, presented in Table 3, support the previously proposed hypothesis [27], with both studies reporting P concentrations in the same order of magnitude (tens of atomic ppm). In the current measurements, the bulk concentration of P in the AR material was around 20–50 ppm, aligning well with the previously measured range. However, the bulk concentration of P in the TA material measured in this work is lower than previously measured, possibly due to the region of interest selected for this study. The bulk composition presented in Table 3 was obtained by measuring P in regions around the selected HAGBs. The proximity to the GB and the P enrichment after thermal aging could be associated with a P depletion zone in the surrounding area, explaining the low P values measured in the supposed “bulk of the grain”. Interestingly, the nominal P concentration for this alloy is around 90 atomic ppm, but both APT studies measured significantly lower bulk amounts of P even before aging. This might indicate some P segregation occurring before thermal aging, potentially at the interface with M_3C -type carbides, as suggested in our

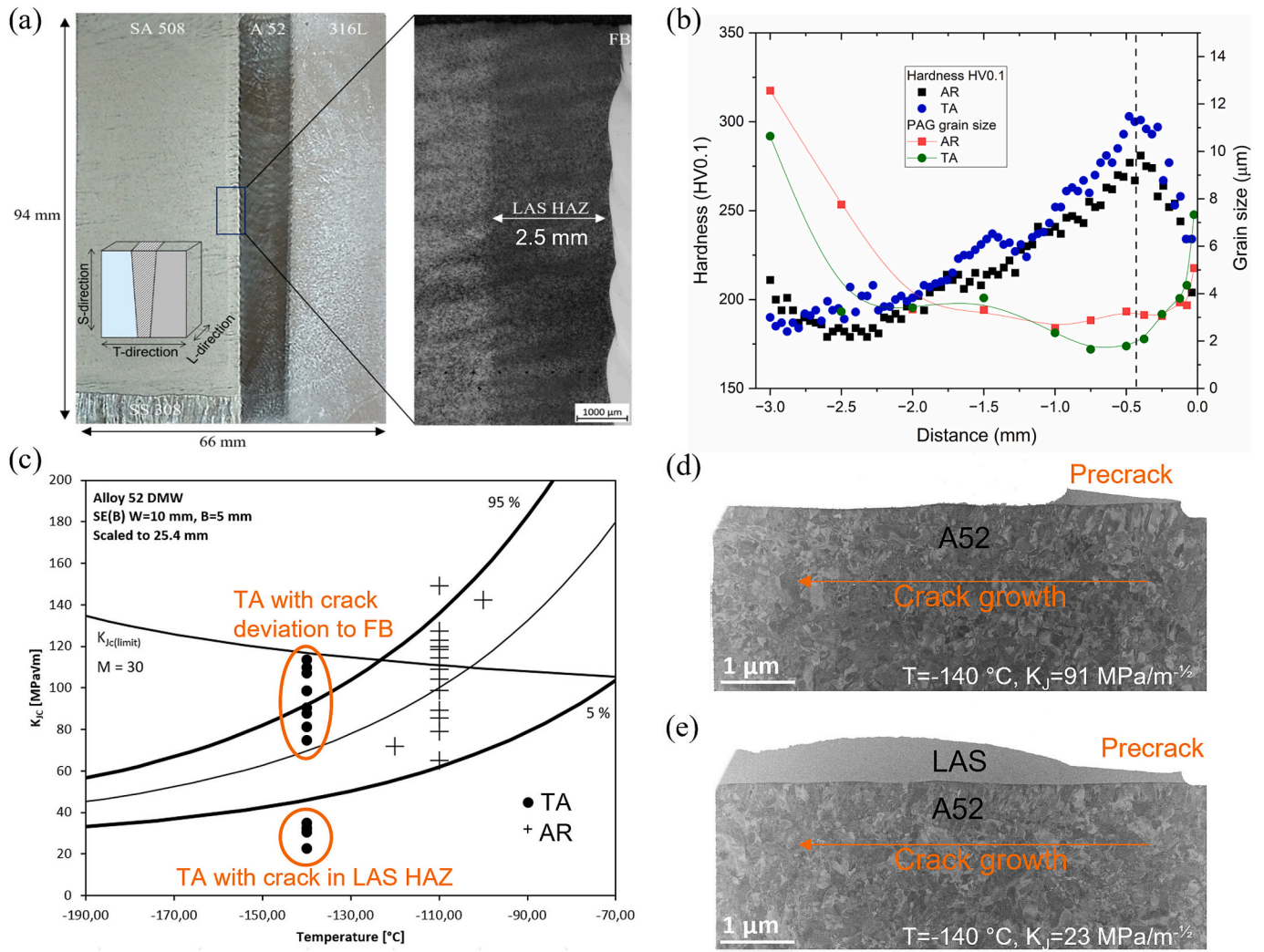


Fig. 2. (a) The investigated narrow-gap GTAW DMW mock-up. (b) The change in micro-hardness and PAG grain size in the HAZ of LAS adjacent to the FB. (c) Summary of the T_0 testing results for AR and TA materials. The representative cross-sectional characterizations of TA T_0 testing specimens (d) with and (e) without crack deviation to the FB. The deviation to the FB resulted in a much higher fracture toughness.

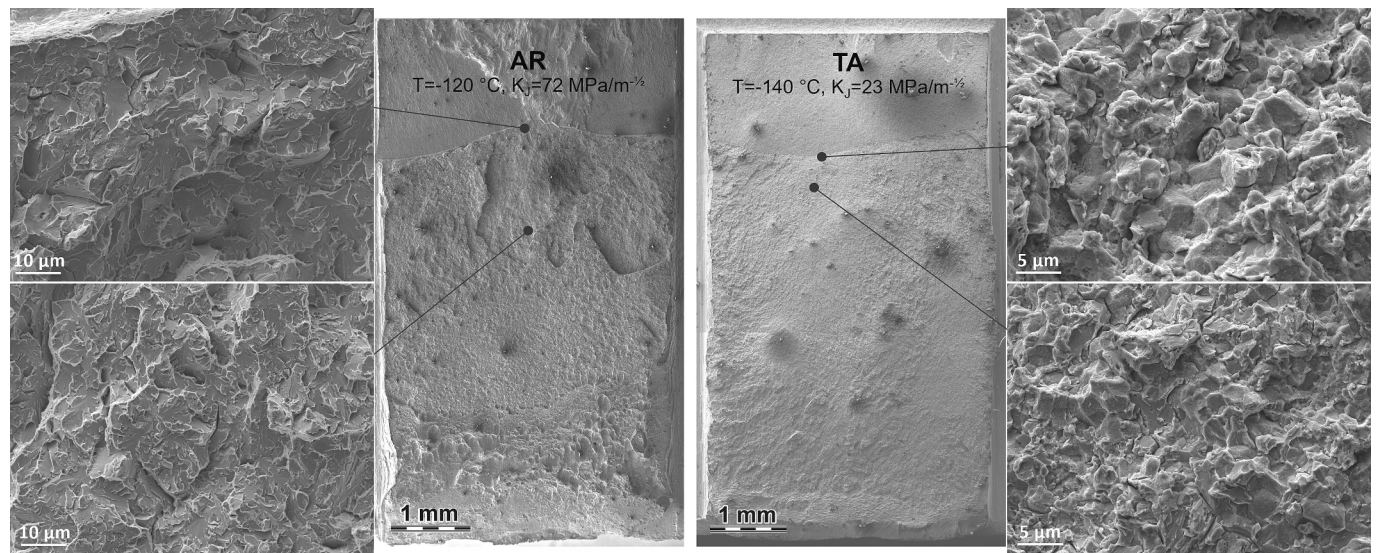


Fig. 3. Representative fracture surfaces of the T_0 fracture toughness specimens in the AR and TA materials.

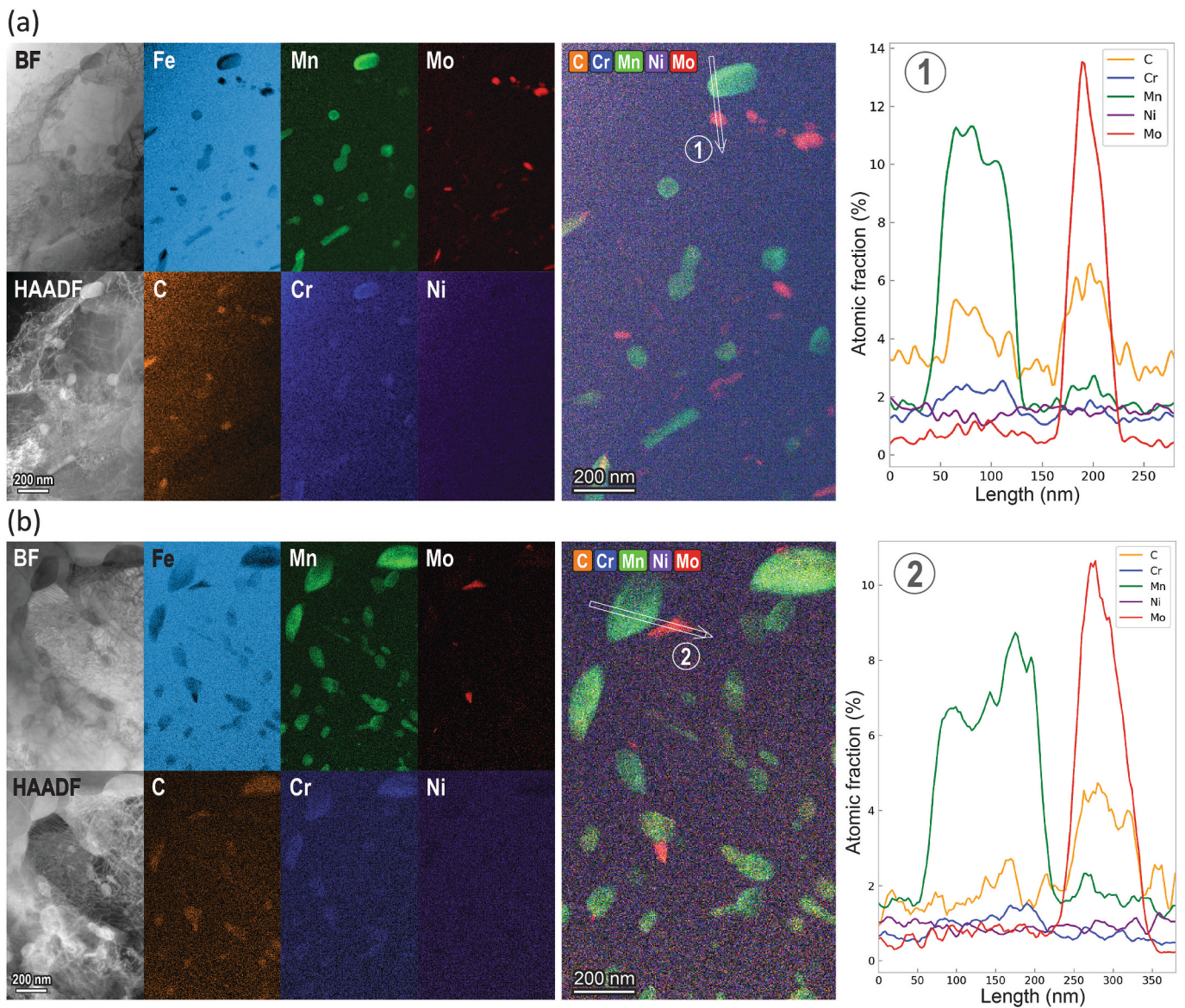


Fig. 4. Carbides in the LAS HAZ of (a) AR and (b) TA materials. The illustrative images are BF images, HAADF images, STEM-EDS elemental maps and extracted line profiles.

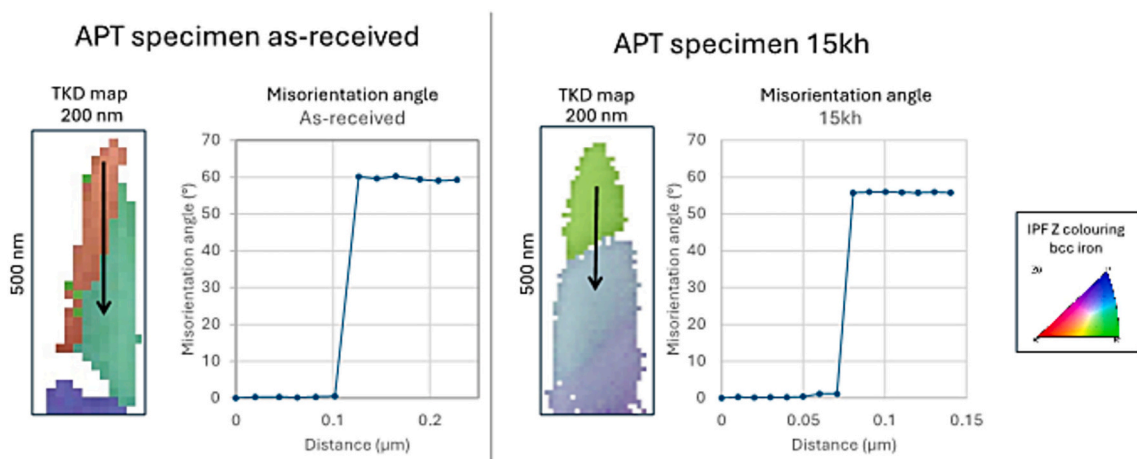


Fig. 5. Representative TKD maps of APT specimens: AR (left) and TA (right) material. High-angle GBs with comparable misorientation were selected.

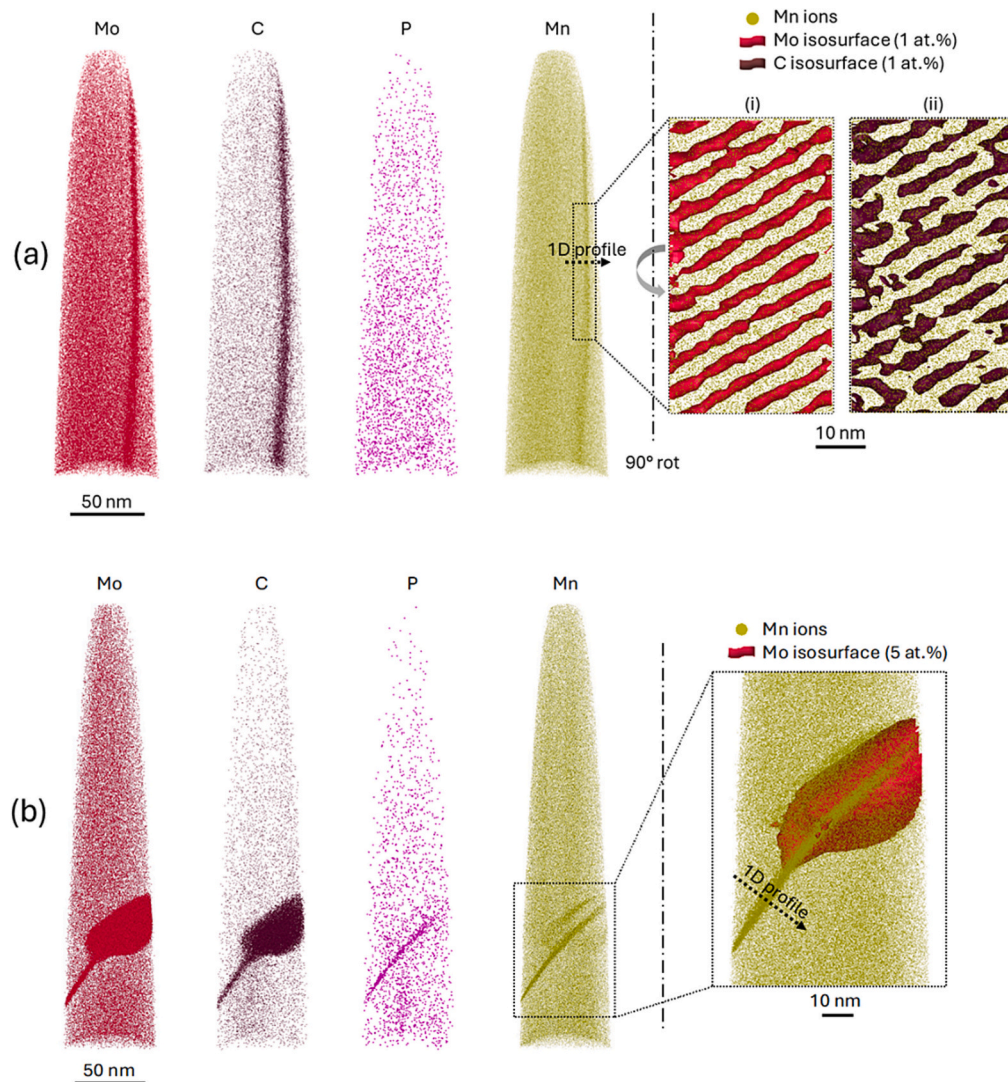


Fig. 6. 3D reconstruction of APT data collected across a HAGB in (a) AR material and (b) TA material. Atom maps of Mo, C, P and Mn are plotted. In (a), segregation of Mo, C and Mn (but no P) is evident from these maps. Segregation of Mo and C follows a pattern fitting enrichment at dislocations or disconnections arranged in a tight array as visible in the boxes to the right (Mo isosurfaces shown in (i), C isosurfaces shown in (ii)). In (b), segregation of Mo, C, P and Mn is evident from these maps. The presence of a Mo-carbide at the GB is evident, a detail of this feature is presented in the box to the right. The location and direction of 1D profiles (see Fig. 7) are indicated by the dashed arrows.

previous work [27].

The APT results on P and Mn segregation at GBs in AR and TA samples suggest that Mn segregation occurs earlier than P, likely due to the remarkably low P content in the material, consistent with previous observations by Kuzmina et al. [36]. In a Fe-9Mn ferritic steel with a P content less than 30 wppm, 10 min of tempering at 450 °C or 600 °C can already induce Mn segregation at GBs, but no P segregation was revealed by APT [36]. Mn segregation leads to a significant reduction in room-temperature impact toughness. The sole segregation of Mn itself in AR and TA samples should contribute to intergranular fracture, a known embrittlement effect of Mn in ferritic and duplex steels [36,37]. In LASs, Mn additions were observed to reduce the resistance to temper embrittlement [38], though the exact mechanisms are under debate. The proposed mechanisms are linked to the effect of Mn on the C activity, potential co-segregation of Mn and P, and the change of P diffusivity [38].

In terms of the interaction between Mn and P, Mn was found to increase P diffusivity and substantially modify the kinetics of P segregation in iron and steels [39,40]. Mn was also observed to enhance P segregation at GBs [41] and accelerate the temper embrittlement in

LASs with the same P content [42]. The simultaneous increase in the segregation levels of P, C, Mn, and Mo in the TA sample, compared to the AR one, is consistent with previous experimental observations [43,44], which is often ascribed to the co-segregation mechanism [45] based on the presumed “attractive interaction” between P and other metal elements [43,46,47]. However, this contradicts ab initio results showing repulsive interactions between P and other transition metals (and with C as well) on a fundamental level [48–50]. As discussed by Suzuki et al. [51], a positive correlation between GB segregation of elements alone cannot prove co-segregation, which may also be attributed to the same GB structure dependence of elemental segregation. It appears that the segregation level at a GB is a competition between local GB structure, alloying element segregation energy, and solute interactions.

Ab initio calculations revealed a stronger P segregation tendency at general GBs (~ -1.0 eV) compared to other transition metals such as Mn, Ni and Mo (~ -0.5 eV) [48–50]. The repulsive interactions of P-Mn/Mo and P-C seem not strong enough to repel P atoms from segregation towards GBs, given the significant number of GB sites available and the low P content. Mai et al. [49] suggested that owing to the repulsive interactions and segregation binding, the enrichment of P

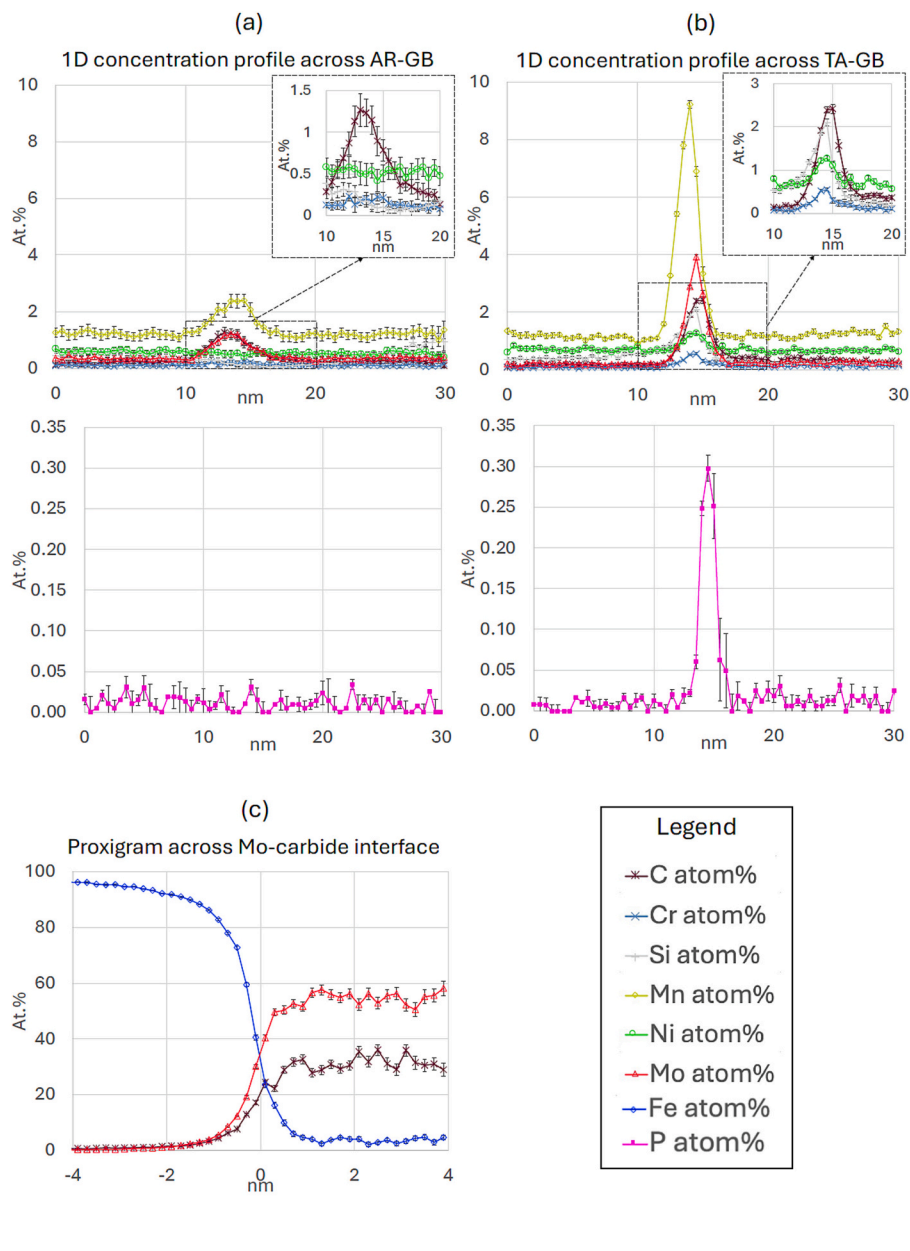


Fig. 7. 1D composition profile across HAGB in the AR material (a), main alloying elements are presented in the top plot while P is presented in the bottom plot. 1D composition profile across HAGB in the TA material (b), main alloying elements are presented in the top plot while P is presented in the bottom plot. Proximity histogram measured across the matrix–carbide interface for the carbide found in the TA material (c).

Table 2
Gibbsian interfacial excess (Γ) calculated for P, C, Mo, and Mn.

Specimen	Γ_P (atoms/nm ²)	Γ_C (atoms/nm ²)	Γ_{Mo} (atoms/nm ²)	Γ_{Mn} (atoms/nm ²)
AR	0.00	2.84	2.43	4.75
TA	0.29	3.01	4.16	12.46

Table 3
Bulk P content of AR and TA materials measured with APT (GBs were excluded).

	P (ppm)	P (wppm)
AR	32 ± 12	18 ± 7
TA	16 ± 4	9 ± 3

upon aging time results in a depletion of prior-segregated cohesion-enhancing solute elements at GBs, which critically influence the P-induced temper embrittlement. However, from the current experimental results, it appears that the segregation of Mn, Mo and C in the AR condition does not effectively repel P atoms from GB segregation upon extended tempering. On the other hand, when the segregation energy of one element is weak, such as Al in a ferritic steel, the strong repulsive interactions of C-Al and B-Al were sufficient to repel Al from GBs [52]. Strong C segregation is usually considered to suppress P-induced embrittlement, based on the C and P site competition mechanism, or the C-enhanced GB cohesion mechanism. Moreover, Mo is considered an element enhancing GB cohesion. In the present work, however, when C segregates to GBs and facilitates the formation and coarsening of carbides, it seems to weaken the resistance to temper embrittlement.

The reduction in brittle fracture resistance of DMWs can arise from inherent local inhomogeneities and increased constraint [22,53]. This

indicates that in the TA specimens, where the crack propagation occurred in the LAS HAZ at a distance of ~0.3–0.5 mm from the FB, the elemental segregation and carbide coarsening upon thermal aging contributed to the low resistance to brittle fracture. Long-term thermal aging at 400 °C for 15,000 h leads to coarsening of carbide precipitates and significant P and Mn segregation to GBs in the HAZ (despite a low bulk P content in modern low-P LASs). The carbide coarsening and segregation result in the presence of local brittle zones (additionally associated with peak nano-hardness and minimal grain size in HAZ), a higher degree of intergranular cracking and a low fracture toughness in the HAZ at a distance of ~0.3–0.5 mm from the FB. The direct observations of significant elemental segregation to GBs and the resultant reduction in fracture toughness in the HAZ are unexpected for modern low-P LASs. This should be considered for evaluating the long-term structural integrity of nuclear components.

4. Conclusions

The following conclusions can be drawn based on the obtained results:

- Long-term thermal aging results in significant P and Mn segregation to GBs (despite a very low bulk P content) and coarsens the carbides in the HAZ, which enhances GB decohesion, leads to local brittle zones, facilitates brittle crack propagation along GBs and ultimately causes a low fracture toughness.
- P can segregate to GBs in LAS after long service times even when its bulk content is very low. Simultaneously, Mn segregates at GBs and contributes to the intergranular fracture. It is observed that when C segregates to GBs and facilitates the formation and coarsening of GB carbides, it weakens the resistance to temper embrittlement.

CRedit authorship contribution statement

Andrea Fazi: Writing – review & editing, Writing – original draft, Methodology, Investigation, Formal analysis, Data curation. **Pedro A. Ferreira:** Writing – review & editing, Methodology, Investigation, Formal analysis, Data curation. **Yanling Ge:** Writing – review & editing, Methodology, Investigation, Formal analysis, Data curation. **Song Lu:** Writing – review & editing, Investigation, Data curation. **Mattias Thuvander:** Writing – review & editing, Resources, Project administration, Methodology. **Zaiqing Que:** Writing – review & editing, Writing – original draft, Resources, Project administration, Methodology, Investigation, Funding acquisition, Formal analysis, Data curation, Conceptualization.

Declaration of competing interest

The authors declare that they have no known competing financial interests or personal relationships that could have appeared to influence the work reported in this paper.

Data availability

The data that support the findings of this study are available from the corresponding author upon reasonable request.

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Title	Effect of Thermal Ageing and Microstructure on Fracture Mechanical Behavior of Ni-based Alloy Dissimilar Metal Welds
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Abstract max. 2000 characters	DMWs are commonly used to join austenitic and ferritic components in the RCPB and can become potential concerns regarding the structural integrity of the nuclear power SSC. In particular, the knowledge on the local strength mismatch at the LAS/nickel-based alloy weld metal interface upon PWHT and during long-term ageing is lacking. The amount of information available in the open literature on the fracture mechanical and microstructural changes occurring at the fusion boundary after buttering, welding, PWHT and long-term ageing is relative limited and often very case dependent. Understanding the fracture mechanical and microstructural appearance, behavior and development of the complex DMW fusion boundary is crucial for both the improvement of nuclear component integrity and to ensure safe long-term operation. In this study, a TVO NG DMW mock-up are studied, thus the test materials are representative of actual power plant components. The project promotes the knowledge transfer, improve the nuclear materials and fracture mechanics competence and strengthen the connections and experience exchange between the Nordic research organizations, universities, industries, authorities, and especially the young generation. The project deals with the structural integrity, long-term operation, and ageing management, which are relevant for both present and future nuclear power plants. The technical results provide a basis for assessment of long-term operation for the Finnish and Swedish nuclear power plants for both the operators and

the regulatory perspectives. Dissemination through the peer reviewed publications and the oral presentations at the international conferences ensured the knowledge exchange in international and Nordic networks.

Key words

Dissimilar metal weld, Alloy 52, fusion boundary, characterization, fracture mechanical test