

Stress test on Norwegian research reactors

NKS Fukushima seminar Stockholm 8th – 9th January 2013

By Atle Valseth



Stress test -initiative

- **Fukushima initiating event (earth quake): 11. March 2011.**
- **Meeting at IFE safety Committee (18 March 2011)**
 - **Topic: Safety at IFE in light of Fukushima accident**
 - **Discussions on 3 scenarios:**
 - Total blackout when reactor in operation.
 - Total blackout during design basis accident
 - Total blackout during beyond design basis accident.
 - **Decision: The thermo hydraulics calculations for both reactors for a new review by the safety committee.**

Re-assessment of thermal hydraulics

- **Reactor coolant thermal hydraulics**
 - Verify cooling by natural heat transfer in all shut down situations
 - Theoretical / experimental
- **Cladding temperature/integrity**
 - Show that local fuel damage would not occur for reactor fuel or experimental fuel.

Assessment both of reactors and spent fuel.

Regulatory requirement (of 18 Sept 2011)

- **IFE to prepare report focusing the following topics:**
 - **Initial events**
 - Earth quake, flooding, power loss.
 - Identification and assessment of potential cliff edge effects
 - **Consequences of loss of safety systems due to the initial events**
 - Assessment of time before fuel failure (reactor and fuel pool) at loss of support function (power)
 - **Emergency preparedness**
 - Evaluation of need for modification of preparedness plans due to stress test
- **Report was sent December 1st 2011**

Initiating events

- **Assessments of initiating events in SAR based on IAEA NS-R-4**
- **Reassessment of initiating events in light of stress test:**
 - **Geophysics, metrology, hydrology.**
 - **Airplane crash a separate assessment (Ongoing)**
 - **Malicious act a separate assessment (ongoing)**
- **Conclusion:**
 - **Assessment in SAR unaltered**

JEEP II



- **Commissioned 1967**
- **«One of a kind»- design**
 - Norwegian design
- **Cooling and Moderation: D₂O**
 - 4 m³
- **Operates at 2 MW, 55 °C and 1 bar**
- **Tank type reactor**
 - -Vessel: Aluminium
- **UO₂ fuel**
 - average 3,4 % enrichment
- **«Steel cylinder» - containment**
 - All reactor systems within containment.

JEEP II results

- **Reactor coolant (D₂O):**
 - Temperature increase at loss of cooling: 17,6 °C (typical) - 47,1°C (max)
 - Worst case; Maximum temperature (100,1 °C) reached after 3,5 days. (Boiling point (D2O): 101,4 °C)
- **Fuel integrity:**
 - Temperature of Fuel cladding (Al) well below safe limits at natural circulation.
- **Spent fuel:**
 - Fuel could in no circumstances be uncovered before 12 days after loss of coolant

Halden Boiling Water Reactor (HBWR)



- **Commissioned 1959**
- **«One of a kind»- design**
 - Prototype
 - Norwegian design
- **Cooling and Moderation: D₂O**
 - 14 m³
- **Operates at 25 MW, 240 °C and 33,6 bar**
- **Tank type reactor**
 - -Vessel: ferritic steel (60 mm) with Stainless steel cladding (5 mm)
- **UO₂ fuel**
 - average 6,6 % enrichment
- **«Rock» - containment**
 - The primary and parts of the secondary system are located in a mountain cage

HBWR results

- **Reactor coolant (D₂O):**
 - Verify reactor cooling/ decay heat removal by natural circulations when shut down from up to 25 MW(t).
 - Normal operation: 18 MW(t)
- **Fuel integrity**
 - Experimental fuel in forced circulation could be uncovered already after 6 minutes.
 - Design modified: For test rigs on forced circulation (approx. 10 out of 30).
- **Spent fuel pool**
 - Worst case: Uncovering of fuel after 7,5 h
 - Design modified: New water line from outside containment.

Emergency preparedness

- **Updated operational procedures**
 - New procedures to cover both loss of power during normal operation and under accident conditions.
 - Loss of instrumentation is a challenging situation, even if power is not needed to maintain safety functions.
- **Review of emergency preparedness during loss of power (ongoing).**
 - Review of available equipment for characterization and communication.
 - Exercise planned.

Regulatory review on report

- NRPA concludes that SAR and Emergency preparedness plans is still valid.
- IFE has identified some areas where there is a need for further assessments and measures
 - NRPA considers non of those to compromise the safety of continued operation.
 - Report by March 9th 2012 and approved thereafter by NRPA.
 - NRPA approval of the stress test report June 29th 2012

Lessons learned

- **External focus at IFE during Fukushima accident**
 - Important of show a pro active attitude.
- **Stress test had a positive effect on safety**
 - Identified gaps.
 - New focus on support infrastructure during emergency situations.
- **Important to verify ability for natural heat transfer.**
 - Low vulnerability for most initiating events
 - Positive value of reassessing thermal hydraulic calculations
- **WENRA stress test requirement as guidance**
 - Guidelines for RR could be useful.